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| ***Poster session I: Metal alloys & novel pathways***  |
|  | **Institution** | **Abstract Title** |
| Ahmed HAIDYRAH | Missouri Univ. , USA | BENDING FATIGUE MINI-SPECIMENTS FOR NUCLEAR MATERIALS |
| Carolina HURTADO-NOREÑA | Argentina | Microstructural characterization of a P91 steel normalized and tempered at different temperatures |
| Changheui JANG | Korea Advanced Institute of Science & Technology, Daejon | The influence of cooling rate from annealing temperature on the microstructure of Haynes 230 |
| Daniel SHEPHERD | NNL, UK | Technology Readiness Level (TRL) Assessment of Cladding Alloys for Advanced Nuclear Fuels |
| Guoping CAO | Univ. Wisconsin, USA | Creep Crack Growth of Nickel Alloy 617 in VHTR Impure Helium and Air |
| Hefei HUANG | SINAP, China | Evolution of microstructure and nanohardness in Hastelloy N alloy after Xe26+ ion irradiation |
| ~~Hyung-ha JIN~~ | ~~KAERI, Rep. Korea~~ | ~~Characterization of microstructural change in ion-irradiated austenitic stainless steel~~ |
| Jan Hoffmann | KIT, Germany | Basic studies on processing of a large scale ODS batch  |
| Joseph (Joe) RASHID | Anatech International Corp., San Diego,USA | A Feasibility Study of Zircaloy-Molybdenum Composite Fuel Rod Cladding Design |
| Karl -Fredrik NILSSON | European Commission, Joint Research Centre, Petten | Assessment of thin-walled cladding tube mechanical properties by segmented expanding mandrel test |
| Kerry ALLAHAR | Boise State University, USA | Spark Plasma Sintering of Oxide Dispersion Strengthened Alloys |
| ~~Kip Findley~~Kristy Tippey | Colorado School of Mines, USA | Processing and Alloying Strategies for Ferritic ODS Alloys and Ferritic-Martensitic Stainless Steels |
| Maria Ines LUPPO | Argentina | Characterization of precipitates in a ASTM A235 Gr P91 welded by means of FCAW process |
| Nancy Lybeck | INL, USA | Development of Yield and Tensile Strength Design Curves for Alloy 617 |
| Pritam CHAKRABORTY | INL, USA | Constitutive Modeling of the Ductile Brittle Fracture Transition in Reactor Pressure Vessel Steels |
| Ruediger KLEIN | Germany | Technical possibilities for the development and testing of new structure materials for a MSR |
| SangHun SHIN | Republic of Korea | A Study on Influence of Liquid Sodium on the Mechanical Properties of Ferritic/Martensitic Steel (HT9) |
| Shigeharu UKAI | Hokkaido University, USA | Unique Fracture Properties by Controlling Crystalline Texture in ODS Ferritic Steels |
| Suk Hoon KANG | KAERI, Rep. Korea | The Effect of Shear Strain Route Variations on the Microstructure Evolution of 316L Stainless Steel |
| Yaqiao WU | Boise State University, USA | Microstructure Characterization of SPS-fabricated ODS Alloys by TEM and APT Techniques |
| Yiren CHEN | ANL, USA | Environmentally Assisted Cracking and Irradiation Embrittlement of CF-8 and CF-8M CASS |
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| ***Poster Session II: Fundamentals & Ceramics*** |   |
| Akira KOHYAMA | Kyoto Uni., Japan | SiC/SiC Fuel Cladding R & D Project, "SCARLET", Status and Future Plan |
| Bilge YILDIZ | MIT, USA | Effect of Niobium on the defect chemistry and corrosion kinetics of tetragonal ZrO2 |
| Cuilan REN | SINAP, China | Temperature effect on primary defect formation in Ni-Fe alloy |
| Darin TALLMAN | USA | A Critical Review of the Oxidation of Ti2AlC, Ti3AlC2 and Cr2AlC in Air |
| Darin TALLMAN | USA | Diffusion Bonding of Zircaloy-4 and Select Mn+1AXn Phases |
| Frank Garner | DLS extreme, USA | Use of ultrasonic techniques to determine the three-dimensional spatial distribution of void swelling and precipitation in thick structural components in response to gradients in neutron flux-spectra and irradiation temperature |
| Hirotatsu KISHIMOTO | Japan | Irradiation Project of SiC/SiC Fuel Pin, "INSPIRE", Status and Future Plan |
| James Marrow | Univ. Oxford, UK | 3D cellular automata finite element (CAFE) modelling and experimental observation of damage in quasi-brittle nuclear materials: ceramic matrix composites and artificial graphite |
| Jean-Louis Courouau | CEA, France | Corrosion by oxidation and carburization in liquid sodium at 550°C of Fe-9Cr steels for sodium fast reactors |
| Joshua KANE | Boise State University, USA | The Oxygen Transfer Mechanism of Graphite Oxidation |
| Lorenzo Malerba | SCK-CEN, Belgium | Object kinetic Monte Carlo simulation of nanostructural evolution under irradiation in Fe-Cr alloys |
| Luca MESSINA | Royal Institute of Technology, Stockholm, Sweden | Combined ab initio-mean field modelling of solute diffusion in bcc Fe-base dilute alloys |
| Mikhail SKLIAR | Uni. Utah, USA | Noninvasive Ultrasonic Measurements of Temperature Distribution and Heat Fluxes in Nuclear Systems |
| Mostafa YOUSSEF | MIT, USA | Mechanistic Modeling of Corrosion and Hydrogen Pickup |
| Sergey Kislitsin | Institute of Nuclear Physics, Kazakstan | SURFACE STRUCTURE OF Ti 0.5Cr0.5N COATINGS AFTER HEAVY IONS IRRADIATION AND ANNEALING |
| Simon MIDDLEBURGH | ANSTO, Australia | High entropy alloys for nuclear applications |
| Simon MIDDLEBURGH | ANSTO, Australia | Effect of Al and Fe Additions to the Solution of Impurities in Beryllium |
| Tomoaki Suzudo | JAEA, Japan | Modeling of He embrittlement of grain boundaries in alpha-Fe |
| Weifeng RAO | INL, USA | Effects of Lattice Defects on the Segregation and Precipitation Behavior in Reactor Pressure Vessel |
| Wen MA | MIT, USA | Effect of Niobium on the initial oxidation of Zirconium based alloys |
| Xianming BAI | INL, USA | Atomistic Studies of the Effects of Strain and GB Character on Oxygen Transport during Zr Corrosion |
| Zhongwen CHANG | KTH, Sweden | Systematic atomistic study of dislocation bias factors in fcc materials |